Examining Transport and Integrated Modeling Predictive Capabilities for Negative-Triangularity Scenarios

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High energy confinement (τ_e) is required for a steady-state fusion reactor [1]. High confinement is typically achieved in positive triangularity (δ) H-mode plasmas. Positive- δ has long been known to improve both confinement and stability. However, the improvement in confinement from the H-mode pedestal typically is accompanied by edge localized modes, which produce large spikes in energy flux that can erode the divertor. Experiments on TCV [2] and DIII-D [3] have shown that L-mode confinement improves dramatically in a negative- δ shape. Two similar DIII-D discharges with matched values for major radius, minor radius, and elongation, but opposite $\delta = -0.4, 0.4$ were compared. The plasmas were heated with up to 10 MW of neutral beam injection (NBI) power and up to 3 MW of electron cyclotron heating (ECH) power and had high levels of confinement.

The earliest modeling of transport of negative- δ suggested that trapped electron modes (TEM) could be suppressed by negative- δ [4]. Gyrokinetic simulations of TCV discharges suggested that trapped electron modes were suppressed in negative- δ compared to the positive- δ counterpart [5]. Exploration of negative- δ showed that negative- δ has reduced transport compared to positive- δ [6], and suggested negative- δ may increase the critical gradient leading to improved transport [7].

The predictive capabilities of TGYRO [8] find that predicted profiles of electron temperature $(T_{\rm e})$, ion temperature $(T_{\rm i})$, electron density $(n_{\rm e})$, and $E \times B$ shear (ω) agree reasonably well in both negative- δ and positive- δ at low auxiliary powers (< 5 MW). TGYRO is analyzed for a positive- δ and a negative- δ plasma heated with $P_{\rm EC}=2$ MW and $P_{\rm NBI}=2$ MW with a fixed boundary condition for the kinetic profiles in these simulations is set to the experimental values at $\rho=0.8$. Figure 1 shows the experimental profile fits and the TGYRO predicted profiles of $T_{\rm i}$, $T_{\rm e}$, $n_{\rm e}$, and ω in positive- δ and negative- δ . Experiments and TGYRO predictions both

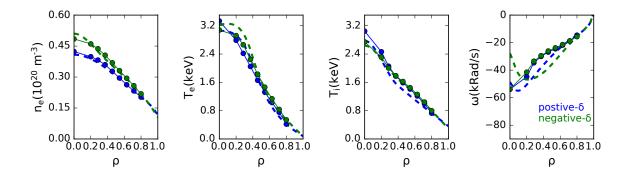


Figure 1: TGYRO prediction (solid) of n_e , T_e , T_i , and ω and experimental profile fits (dashed) in NBI+ECH heated plasmas in positive- δ (blue) and in negative- δ (green).

have higher values for $n_{\rm e}$ and $T_{\rm e}$ are observed in negative- δ compared to positive- δ . The $T_{\rm e}$ profile is slightly underpredicted for both positive- δ and negative- δ . The $T_{\rm i}$ profile is slightly overpredicted (\sim 10%) for positive- δ , while it is almost perfectly predicted in negative- δ .

Analysis of the turbulent transport using the quasilinear gyro-Landau fluid code TGLF [9] is performed. TGLF predicts a large difference in particle flux (Γ) between positive- δ and negative- δ as a function of temperature gradient scale length a/L_{T_e} , as shown in Figure 2. Here, a is the minor radius, $1/L_{T_e} = \partial T_e/\partial r/T_e$. As a/L_{T_e} is increased, the predicted Γ for the negative- δ scenario becomes negative. This is opposite to the predicted flux in the positive- δ scenario where Γ increases as a/L_{T_e} moves away from the experimental gradient. This difference in particle transport behavior could possibly lead to better con-

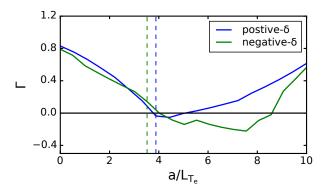


Figure 2: TGLF-SAT0 predicted Γ (10²⁰ m⁻² s⁻¹) at $\rho = 0.6$ for a positive- δ and negative- δ ECH only plasma plotted vs. a/L_{Te} . The vertical dashed lines indicate the experimental scale lengths.

finement for negative- δ with negative- δ having better higher experimental n_e and T_e in Figure 2. Core-pedestal modeling shows confinement and β_N improve at negative δ and at positive δ . Similar parameters to the DIII-D experiment of $\beta_N = 2$, $n_{\rm e,ped} = 0.35 \times 10^{20}$ m⁻³, $\kappa = 1.6$, $B_t = 2$ T, and $I_p = 0.8$ MA are used. Modeling is done using the STEP [10] module in OMFIT [11], which iterates between TGYRO[8], ONETWO [12], and CHEASE [13]. TGYRO is used to predict kinetic profiles. ONETWO is used to predict the current evolution. CHEASE is used

to to ensure Grad-Shafranov is satisfied with a Miller geometry The current is evolved assuming the current fully penetrates to steady-state from neoclassical resistivity. An additional radially constant current diffusion is applied where q < 1 to raise the on-axis q just above unity. A radially uniform $Z_{eff} = 1.7$ is used. Fixed Gaussian sources of electron heating are used, one located at $\rho = 0.6$ and the other at $\rho = 0.0$ to represent electron cyclotron heating. The pedestal boundary condition is taken to be the EPED [14] prediction with $\beta_N = 2$ shown in Figure 3. The pedestal height increases monotonically with increasing δ . While H-mode has rarely been observed experimentally for strong negative- δ potentially prevented due to ballooning modes [15], we still use EPED to set the edge conditions for negative triangularity as EPED has been shown to accurately predict negative- δ plasma down to $\delta = -0.2$ [16]. Using EPED below $\delta = -0.2$ gives a reduction in edge confinement at negative- δ without needing different edge models for negative- δ and positive- δ . TGLF-SAT0 is used in TGYRO with settings for electrostatic and using the $E \times B$ quench rule.

The STEP predicted confinement improvement at negative- δ is predicted to be stronger at high power densities and with strong electron heating sources. Figure 3 shows STEP predicted β_N vs triangularity for two injected powers: 10 MW and 20 MW, and NBI only and a 50/50 mix of NBI+ECH. At $P_{aux} = 10$ MW for both heating types, modest improvements in β_N are predicted with simulations when δ is decreased from $\delta = -0.2$ to $\delta = -0.6$. For the 50/50 mix of NBI+ECH heating, the STEP prediction of β_N becomes

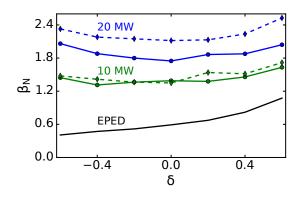


Figure 3: STEP predicted β_N NBI only (dashed) and NBI+ECH (solid) heating $P_{aux} = 10$ MW (blue) and $P_{aux} = 20$ MW (green) with EPED boundary (black).

U-shaped at $P_{aux} = 20$ MW with positive- δ with $\beta_{N,\delta=-0.6} \sim \beta_{N,\delta=0.6}$. The increase in β_N at negative- δ compared to positive- δ is stronger for NBI+ECH than with NBI only. However, all values of β_N are lower than the NBI only prediction.

The transport and integrated modeling predictive capabilities of the negative- δ scenario are examined in which TGYRO predicts stronger $n_{\rm e}$ and $T_{\rm e}$ gradients in negative- δ compared to positive- δ , consistent with the experiment. TGLF predicts that increasing electron temperature gradient scale length reduces the particle transport in negative- δ . This suggests that electron heating may not degrade core plasma particle confinement the way positive- δ does. The predicted confinement improvement at negative- δ from core-pedestal modeling is stronger in high

power electron heated plasmas.

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